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## EDITED BY

Giuseppe Verde,  
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## REVIEWED BY

Nicola Colonna,  
National Institute for Nuclear Physics of  
Bari, Italy  
Ian Thompson,  
Lawrence Livermore National Laboratory  
(DOE), United States

## \*CORRESPONDENCE

Vlad Avrigeanu,  
✉ vlad.avrigeanu@nipne.ro

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# Consistent assessment of neutron-induced activation of $^{93}\text{Nb}$

Vlad Avrigeanu\* and Marilena Avrigeanu

Horia Hulubei National Institute for Physics and Nuclear Engineering (IFIN-HH), Magurele, Romania

A concurrent assessment of all measured excitation functions for various reactions induced by neutrons on  $^{93}\text{Nb}$ , in addition to the results of TENDL-2021 and default parameters in TALYS-1.96, is given in this work. We use consistent parameter sets that were formerly obtained or validated by the analysis of other independent data, while no empirical rescaling factors of  $\gamma$  and/or neutron widths have been used. The correlation between the measured error bars of the primary data providing the consistent input parameters and the final uncertainty bands of the calculated results have been pointed out. At the same time, a proper account in this work of all available data for competitive reaction channels prevented compensation effects of less accurate model parameters. Remaining questions and the need for additional measurements are emphasized.

## KEYWORDS

nuclear reactions, cross-sections, nuclear models, optical potential, nuclear level density, model calculation uncertainty bands

## 1 Introduction

The interest in niobium has followed its application in structural materials of nuclear reactors, activation monitor in reactor dosimetry, 14 MeV neutron flux determination, and element of superconductor alloys in fusion reactors. A consequent large body of experimental data for neutron interactions with the  $^{93}\text{Nb}$  nucleus also guided its use as a “sample problem” [1] in statistical Hauser–Feshbach (HF) [2] and pre-equilibrium emission (PE) [3] model calculations [4–10]. However, rather sparse measured data, for neutron-induced reactions on this only Nb natural isotope are yet pointing out a need of more accurate measurements to settle its evaluation [11–13].

Moreover, the concurrent assessment of all measured excitation functions for various reactions induced by neutrons on  $^{93}\text{Nb}$ , in addition to the results of using default parameters within the worldwide used computer code TALYS-1.96 [14] and the currently related TENDL-2021 evaluation [15], remains an actual goal of further nuclear model analysis. This demand has been confirmed within a recent assessment of the  $\alpha$ -particle emission in neutron-induced reactions on Zr, Nb, and Mo stable isotopes [16]. Furthermore, we look for a proper account of all available data for competitive reaction channels to prevent compensation effects of less accurate model parameters.

Consequently, we have paid more attention to the use of consistent parameter sets that are formerly obtained or validated by the analysis of other independent data [17]. Thus, no empirical rescaling factors of  $\gamma$  and/or neutron widths are involved. On the other hand, detailed analyses based on consistent input parameter sets are needed to eventually improve the global parameters for involvement in large-scale evaluations. Such a case has been that of the  $\alpha$ -particle optical model potential [18] adopted as the corresponding default option of

**TABLE 1** Low-lying levels number  $N_d$  up to excitation energy  $E_d^*$  [31] used in SM calculations, and  $N_d$  and  $s$ -wave nucleon-resonance spacings<sup>a</sup>  $D_0^{\text{exp}}$  (with uncertainties given in units of the last digit in parentheses) in the energy range  $\Delta E$  above the separation energy  $S$ , for target nucleus g.s. spin  $I_0$ , fitted to obtain BSGF level-density parameter  $a$  and g.s. shift  $\Delta$  (for a spin cut-off factor corresponding to a variable moment of inertia [32] between half and 75% of the rigid-body value, from g.s. to  $S$ , and reduced radius  $r_0 = 1.25$  fm).

| Nucleus          | $N_d$ | $E_d^*$ (MeV) | Fitted level and resonance data |               |                                |       |                          | $a$ (MeV <sup>-1</sup> ) | $\Delta$ (MeV)      |
|------------------|-------|---------------|---------------------------------|---------------|--------------------------------|-------|--------------------------|--------------------------|---------------------|
|                  |       |               | $N_d$                           | $E_d^*$ (MeV) | $S + \frac{\Delta E}{2}$ (MeV) | $I_0$ | $D_0^{\text{exp}}$ (keV) |                          |                     |
| <sup>89</sup> Y  | 26    | 3.630         | 26                              | 3.630         | 11.478                         | 4     | 0.106 (35) <sup>b</sup>  | 8.90                     | 0.94                |
| <sup>90</sup> Y  | 30    | 2.366         | 29 (2)                          | 2.327         | 6.857                          | 1/2   | 3.7 (4)                  | 9.23 (15) (10)           | -0.32 (5) (2)       |
| <sup>92</sup> Zr | 42    | 3.500         | 54 (2)                          | 3.725         | 8.647                          | 5/2   | 0.55 (10)                | 9.67 (27) (25)           | (0.79 9) (6)        |
| <sup>93</sup> Zr | 29    | 2.391         | 29 (4/2)                        | 2.391         | 6.785                          | 0     | 3.5 (8)                  | 10.66 (43) (34)          | 0.12 (10) (0/3)     |
| <sup>90</sup> Nb | 31    | 1.692         | 31                              | 1.692         |                                |       |                          | 9.2 (4/2)                | -1.02 (8) (13/7)    |
| <sup>91</sup> Nb | 29    | 2.660         | 29 (6/0)                        | 2.660         |                                |       |                          | 9.3 (4/2)                | 0.04 (12/6) (0/6)   |
| <sup>92</sup> Nb | 41    | 1.851         | 41 (2)                          | 1.851         |                                |       |                          | 9.6 (4/2)                | -0.92 (12/7) (9/3)  |
| <sup>93</sup> Nb | 35    | 1.784         | 35 (2)                          | 1.784         |                                |       |                          | 9.9 (4)                  | -0.80 (14/9) (11/5) |
| <sup>94</sup> Nb | 48    | 1.281         | 48 (2)                          | 1.281         | 7.232                          | 9/2   | 0.094 (10)               | 10.75 (16) (12)          | -1.29 (5) (1)       |

<sup>a</sup>RIPL-3 [33] if not otherwise mentioned.

<sup>b</sup>Ref. [53].

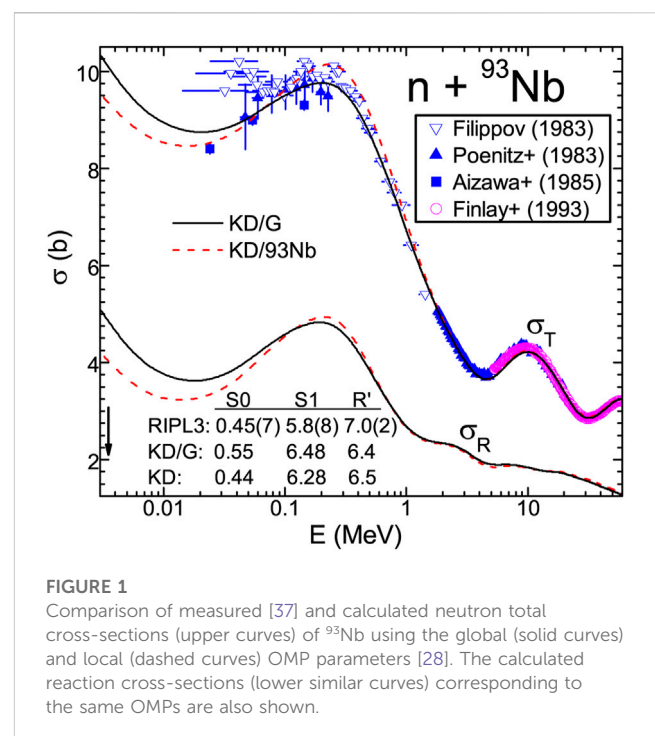
TALYS, whose suitable setup and confirmation [19–22] made use of consistent parameter sets.

Nonetheless, the accuracy of the independent data formerly involved in the setup of a consistent parameter set entirely determines the appropriateness of these parameters and, finally, the uncertainties of the calculated reaction cross-sections. Thus, the use of consistent parameters leads to increased accuracy of model calculations, but only within the limits of actual knowledge about the primary data triggering the input parameters. This is why, in this work, we look for the definite relation between the measured error bars of the primary data providing the consistent input parameters, the corresponding limits of these parameters, and, finally, the uncertainty bands of the calculated results. On the other hand, the use of no empirical rescaling factors completes the consistency of both input parameters and calculation results by agreeing with something previously obtained and always behaving in a similar way.

This work completes Ref. [16] with reference to the target nucleus <sup>93</sup>Nb, and the only additional HF+PE model parameters for neutron-induced reactions on this nucleus are given in Section 2. Comparison of HF+PE results and all available related data are discussed in Section 3. Conclusions are finally given in Section 4.

## 2 Nuclear models and parameters

The HF+PE reaction analysis and collective inelastic scattering cross-section assessment have been carried out using the same models, codes [14, 23–25], and previous local approaches [16, 26]. Moreover, the same parameters, consistently established or validated by means of distinct data, have been involved for the 1) back-shifted Fermi gas (BSFG) [27] nuclear-level density (NLD), 2) particle transmission coefficients through optical model potentials (OMPs) [18, 28], and 3)  $\gamma$ -ray transmission coefficients through radiative strength functions (RSFs) [29, 30]. The same NLD and OMP parameters have been used in the framework of

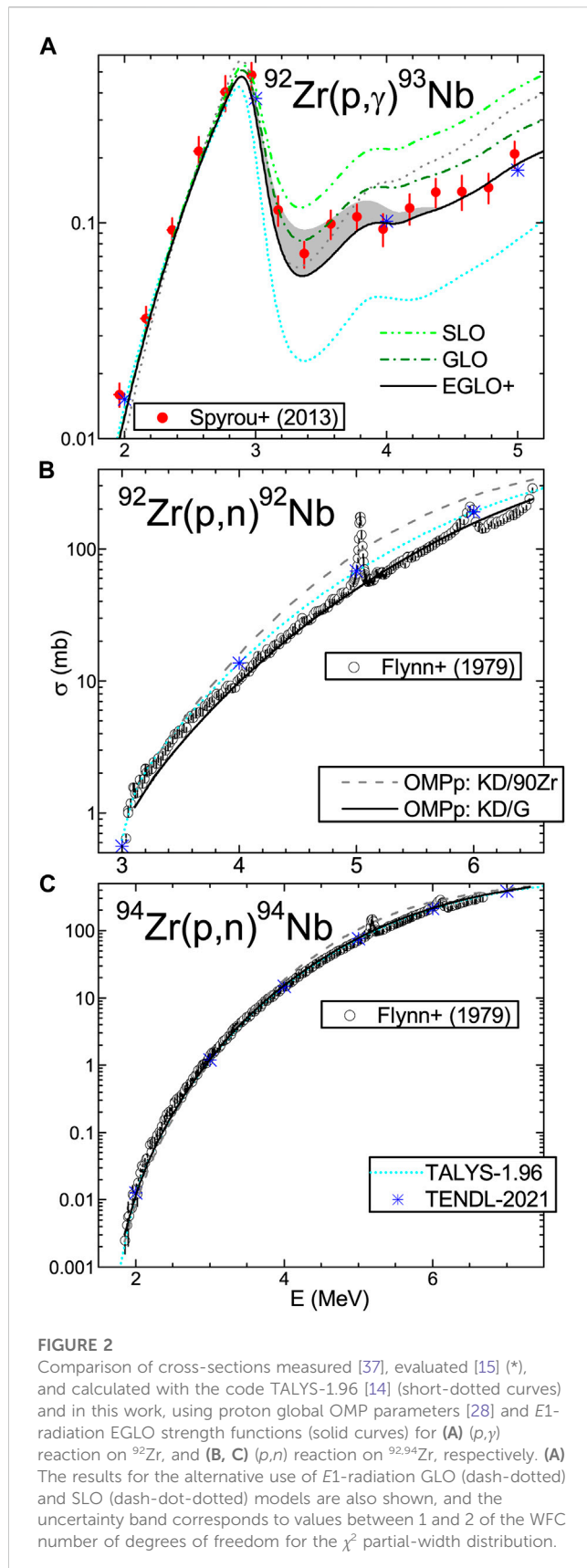


**FIGURE 1**

Comparison of measured [37] and calculated neutron total cross-sections (upper curves) of <sup>93</sup>Nb using the global (solid curves) and local (dashed curves) OMP parameters [28]. The calculated reaction cross-sections (lower similar curves) corresponding to the same OMPs are also shown.

the HF+PE models, with additional pickup direct-reaction (DR) contribution to the  $\alpha$ -particle emission.

NLD parameters for the main reaction products concerned in the present work in addition to Ref. [16] are given in Table 1, including the corresponding uncertainties. The last ones correspond first to the fit of the  $s$ -wave nucleon-resonance spacing  $D_0^{\text{exp}}$  [33] in  $\Delta E$  energy range [34] above separation energy  $S$ , for the target-nucleus ground state (g.s.) spin  $I_0$ . The additional uncertainty of the fitted number of low-lying levels  $N_d$ , up to excitation energy  $E_d^*$  [31], led to enlarged BSGF model parameter uncertainties (second pair of



brackets in Table 1). The smooth-curve method [35] was applied for nuclei without resonance data, and an average of fitted  $a$ -values of nearby nuclei was used to obtain only the  $\Delta$  values by fitting low-

lying discrete levels alone. Larger uncertainties of averaged  $a$ -values have resulted in this case, due to the spread of the former ones. Finally, the NLD-parameter uncertainties have been used to illustrate the NLD effects on calculated cross-sections (Section 3).

The neutron transmission coefficients have been obtained using the global and local OMPs of Koning and Delaroche [28], and their additional analysis by means of the SPRT method [36] within the energy range of this work. Thus, these OMP results for the  $s$ - and  $p$ -wave neutron strength functions  $S_0$  and  $S_1$ , respectively; the potential scattering radius  $R'$ ; and the energy dependence of the neutron total cross-section  $\sigma_T(E)$  are compared in Figure 1 with the available data [32, 33, 36]. We found that the global parameter set [28] provides a particularly better agreement of the measured and calculated  $\sigma_T(E)$  in comparison to the related local parameter set within this energy range. So, we used the aforementioned global parameter set [28]. Nevertheless, the corresponding changes also shown in Figure 1 for the reaction cross-sections  $\sigma_R(E)$ , of notable importance around 1 MeV for the competition between the neutron and charged-particle decay of excited compound nuclei (CNs), have no importance.

Moreover, the same OMP parameters provided the collective inelastic scattering component within the distorted-wave Born approximation (DWBA) method, the code DWUCK4 [24], and the deformation parameters [4] of the first  $2^+$  and  $3^-$  collective states. It goes up to  $\sim 7\%$  of  $\sigma_R$  for incident energies around 4 MeV and then decreases slightly below 5% at energies above 20 MeV. A corresponding decrease in  $\sigma_R$  has been then taken into account within the PE+HF analysis of various reaction channels.

The proton transmission coefficients of a similar alternative have been solved by the usual analysis of the  $(p,n)$  cross-sections at energies ( $\geq 3$  MeV) where this reaction channel has cross-sections close to the optical potential  $\sigma_R$ . However, because only earlier measured data have been available for this reaction on neighboring Zr isotopes (Figure 2B,C), more recent data on the  $(p,\gamma)$  reaction have been used below the  $(p,n)$  reaction effective threshold, where its cross-sections are yet to come close to  $\sigma_R$  values. Then, the comparison of measured and calculated cross-sections shown in Figure 2A led to the further involvement in this work of the global proton OMP [28].

The  $\alpha$ -particle transmission coefficients corresponding to the  $\alpha$ -particle OMP [18] have just been proved to be well suited for  $\alpha$ -emission in neutron-induced reactions in the mass range  $A \sim 90$  including  $^{93}\text{Nb}$  nucleus [16]. A still-open question to be answered in this work may concern only the role of the uncertainties related to the competitive reaction channels (Section 3).

The  $\gamma$ -ray transmission coefficients involve RSFs, which unfortunately have no confident parametrization despite the widespread systematics [30, 37] performed even recently. So, we have adopted the giant dipole resonance (GDR) parameters of Kopecky and Uhl [39] within the former Lorentzian (SLO) [40], the generalized Lorentzian (GLO) [39], and the enhanced generalized Lorentzian (EGLO) [41] models for the electric-dipole RSF. The additional  $M1$  upbend parameters found recently to describe the RSF data for  $^{92,94}\text{Mo}$  nuclei, i.e., the middle resonance parameters given in Table II of Ref. [42], have been used for  $^{93,94}\text{Nb}$  nuclei too.

Comparison of the results obtained within the aforementioned models for the E1-radiation in Figure 2A showed that the EGLO model has led to an enhanced RSF description over the whole energy range of the measured  $(p,\gamma)$  cross-sections [43]. Furthermore, an eventual uncertainty band of the calculated results corresponding to

the values between 1 and 2 for the number of degrees of freedom for the  $\chi^2$  partial-width distribution within Moldauer's width fluctuation correction (WFC) [44] has fully covered the scattered experimental data around the excitation function minimum just above the  $(p,n)$  reaction threshold.

A similar analysis to be shown hereafter for the neutron capture on  $^{93}\text{Nb}$  comes to the same conclusion concerning the average  $s$ -wave radiation widths  $\Gamma_\gamma$ . Thus, the SLO and GLO models have led to larger captured cross-sections and  $\Gamma_\gamma$  values in comparison to the measured data [33, 36].

The *pre-equilibrium emission* account within the geometry-dependent hybrid (GDH) model [45], generalized through the inclusion of the angular momentum and parity conservation [46] and knockout  $\alpha$ -particle emission, has a main parameter of pre-formation probability  $\varphi$  [3]. Its suitable values have been recently obtained [16] by the analysis of several energy spectra measurements around the incident energy of 14 MeV. On the other hand, the GDH intra-nuclear transition rates were calculated also using the aforementioned imaginary OMP parameters within the local density approximation ([45] and Refs. therein). Local-density Fermi energy was similarly related to various partial waves and a central-well Fermi energy value  $F = 40$  MeV.

Moreover, the revised version of the advanced particle-hole-level densities (PLD) [47, 48] was used with the Fermi-gas energy dependence of the single-particle level (s.p.l) density [49]. The s.p.l. density  $g_\alpha$  of the pre-formed  $\alpha$ -cluster, which behaves like an exciton [3, 45], has been related to the level-density parameter  $a$  through the usual equidistant spacing-model relation  $g = (6/\pi^2)a$ .

The *DR pickup mechanism* has also been used to describe the  $\alpha$ -particle emission in neutron-induced reactions, similar to recent analyses in the mass ranges  $A \sim 60$  [21, 22] and  $A \sim 90$  [16] including  $^{93}\text{Nb}$ . However, the lack of measured angular distributions for the  $(n, \alpha)$  reaction again made impossible the current DWBA assessment of related pickup cross-sections using spectroscopic factors (SFs) obtained by the analysis of  $\alpha$ -particle angular distributions.

Thus, one-step reaction has also been considered through the pickup of  $^3\text{He}$  clusters. However, the "spectator model" [50, 51] was used for the pair of transferred protons in  $^{93}\text{Nb}(n, \alpha)^{90}\text{Y}$  reaction, with the corresponding SFs given by Glendenning [52]. Then, the SFs for the picked neutron were obtained on account of their similarity with the picked-proton SFs found by the analysis of the  $\alpha$ -particle angular distributions of the pickup reaction  $^{91}\text{Zr}(t, \alpha)^{90}\text{Y}$  toward the same residual nucleus [16]. The DWBA formalism with prior-form transition amplitudes and finite-range interactions within the code FRESKO [25] have been used. Last, the description of the  $\alpha$ -particle angle-integrated energy distributions around 14 MeV induced by neutrons on  $^{93}\text{Nb}$  has validated the  $\alpha$ -particle's both PE and DR components [16].

### 3 Results and discussion

The  $(n, \gamma)$  excitation-function analysis is of general interest due to the significant activation of various isomeric states by neutrons incident on  $^{93}\text{Nb}$ . The former validation of the  $\gamma$ -ray transmission coefficients has confirmed the adopted NLD spin cut-off factors, which then trigger all isomeric cross-sections. Therefore, a good

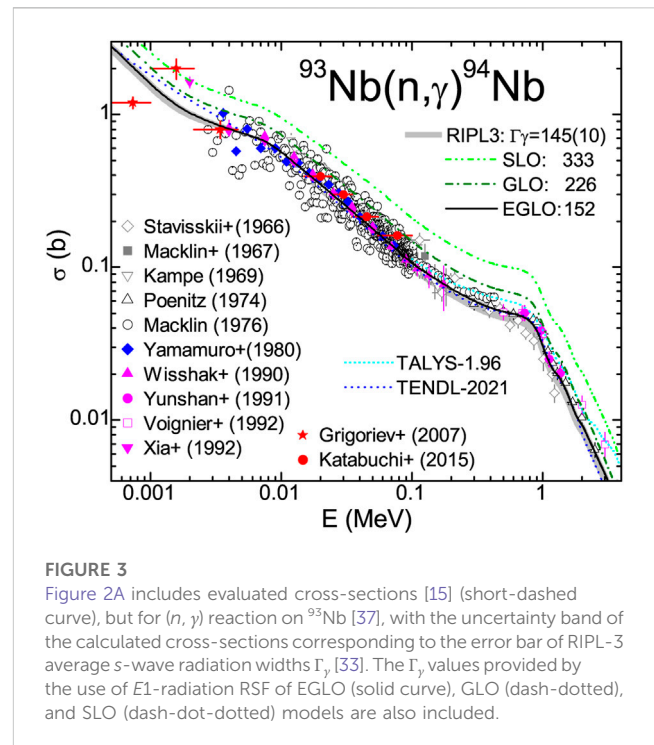


FIGURE 3

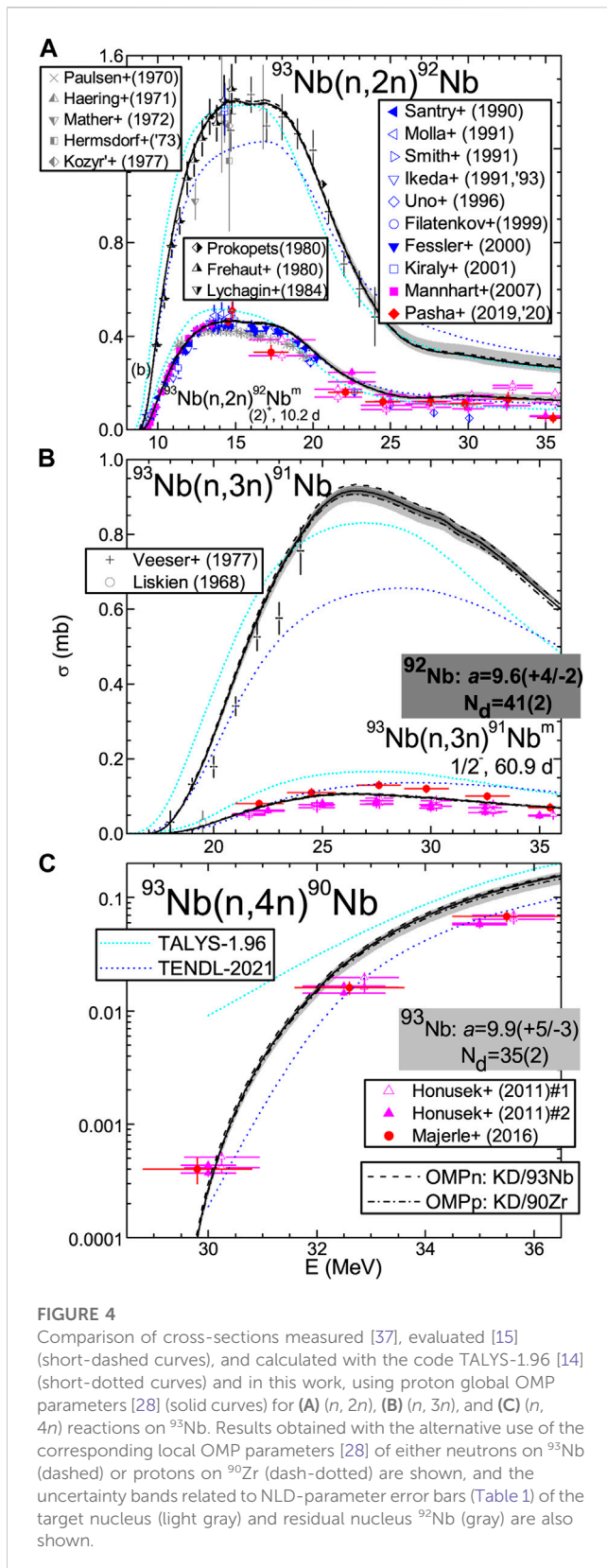
Figure 2A includes evaluated cross-sections [15] (short-dashed curve), but for  $(n, \gamma)$  reaction on  $^{93}\text{Nb}$  [37], with the uncertainty band of the calculated cross-sections corresponding to the error bar of RIPL-3 average  $s$ -wave radiation widths  $\Gamma_\gamma$  [33]. The  $\Gamma_\gamma$  values provided by the use of  $E1$ -radiation RSF of EGLO (solid curve), GLO (dash-dotted), and SLO (dash-dot-dotted) models are also included.

agreement between the more recent experimental data and the calculated results shown in Figure 3 is particularly important.

At the same time, it should be pointed out that a good agreement has also been obtained for the related RIPL-3 average  $s$ -wave radiation width, even within quite a narrow error bar [33]. Actually, the narrow uncertainty band of the calculated cross-sections corresponding to the  $\Gamma_\gamma$  limits (Figure 3) is close to the more recently measured data. On the other hand, replacing the EGLO model for the  $E1$ -radiation RSF with GLO and especially SLO models, a notable overestimation follows both for the calculated excitation function and  $\Gamma_\gamma$  values. Thus, the use of the GLO model leads to increased cross-sections and  $\Gamma_\gamma$  values by  $\sim 37\%$  and  $\sim 50\%$ , respectively, around the incident energy of 1 MeV, while the SLO model corresponds to over two times larger values.

The  $(n, 2n)$  reaction analysis has the shortcoming of no total cross-section data available within the last 40 years. Fortunately, there are recent measurements for the isomeric cross-sections corresponding to the  $2^+$  state of  $^{92}\text{Nb}$  nucleus at 136 keV, as shown in Figure 4A. First, the agreement between the calculated and recently measured isomeric cross-section at the incident energy of  $\sim 14$  MeV, i.e., on the flat maximum of this excitation function should be noted. Second, it is also notable that the concurrent suitable account of both the isomeric state and total  $(n, 2n)$  excitation functions is obtained. At the same time, the TALYS-1.96 results are in better agreement with only the latter data, while TENDL-2021 fits to a greater extent the former excitation function. There is thus support for the neutron OMP and the NLD spin distribution corresponding to the aforementioned spin cut-off factors.

The size of the NLD effects on calculated cross-sections closely follows the NLD-parameter uncertainty related to the error bars of the fitted  $D_0^{\text{exp}}$  and low-lying levels (Table 1). Moreover, the spread



of the formerly fitted LD parameters  $a$  is also quite important within the smooth-curve method involved for nuclei without the resonance data. It determines the adopted limits of the average  $a$ -values, given in Table 1 as well, and finally the corresponding uncertainty bands of

the calculated excitation functions. On the other hand, the particular minimum of the  $a$ -systematics for nuclei around the magic number  $N = 50$ , as the Nb residual nuclei, has eventually led to adopted upper deviations from the average  $a$ -value, which are larger than the lower deviations, e.g.,  $a = [9.2 \pm (0.4/0.2)] \text{ MeV}^{-1}$  for  $^{92}\text{Nb}$  nucleus (Table 1).

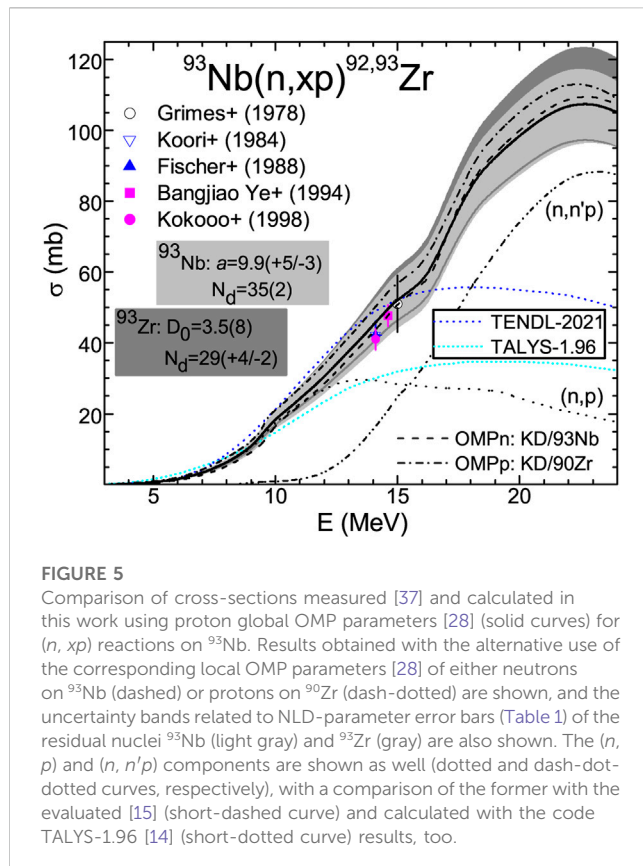
Thus, Figure 4A shows the bands related to the NLD-parameter limits of the residual nuclei  $^{92,93}\text{Nb}$ , which deserve the following comments. The wider uncertainty bands are related to the limits of the NLD parameters for  $^{93}\text{Nb}$  nucleus populated through the excited nucleus  $^{94}\text{Nb}$  decay by neutron emission. However, even these bands, rather similar for the total and isomeric cross-sections, are larger than  $\sim 1\%$  only for incident energies above 20 MeV. Then, it becomes either higher or lower than the calculated cross-sections at the incident energy of 37 MeV by  $\sim 14\%$  and  $\sim 8\%$ , respectively. At the same time, the uncertainty bands corresponding to the limits of the NLD parameters for the residual nucleus  $^{92}\text{Nb}$  of the  $(n, 2n)$  reaction are overall below 1%.

The changes in the calculated  $(n, 2n)$  reaction cross-sections due to an eventual use of the neutron local OMP [28] have also been only from about 1% at the excitation function maximum to  $\sim 2\%$  around 35 MeV. The replacement of the aforementioned proton local OMP [28] has led to the changes below 1% at all incident energies. One may thus conclude that there is quite a low model-parameter dependence of the calculated  $(n, 2n)$  cross-sections, particularly below the incident energy of 20 MeV, for the case of consistent parameters determined formerly by the analysis of independent data.

The  $(n, 3n)$  reaction analysis has shown quite similar results in Figure 4B, particularly for the agreement with the recently measured isomeric cross-sections. Uncertainty bands similar to the case of the  $(n, 2n)$  reaction have been obtained, with only about 1% larger limits. Comparable changes have followed the use of either neutron or proton local OMPs [28]. The latter now becomes distinct from those related to the global proton OMP for  $^{93}\text{Zr}$ , for the total  $(n, 3n)$  excitation function, but close to it. On the other hand, the uncertainty bands corresponding to the NLD parameters of the residual nucleus  $^{91}\text{Nb}$  have widths within 3% of the calculated excitation functions and thus cannot be distinguished.

The  $(n, 4n)$  reaction-calculated cross-sections shown in Figure 4C are in good agreement with the measured data around the incident energy of 30 MeV but overestimated around 35 MeV. This disagreement remains even beyond the uncertainty band related to the NLD parameters of the residual nucleus  $^{90}\text{Nb}$  within the  $(n, n')$  reaction. Despite the width of this band going from 14% to 23%, for energies from 30 to 37 MeV, it is yet above the measured data by 2–3 times, while the threshold energy for the  $(n, 5n)$  reaction is above 39 MeV. Lower changes due to the use of local OMPs [28] for either neutrons or protons, of only about +2% and  $-6\%$ , respectively, cannot improve the measured data description. Eventually, a less suitable PE energy-dependence account at these higher energies may explain this variance and should be a concern within further studies.

The  $(n, xp)$  reaction analysis has to overcome the shortcoming of no measured excitation function. Only several angle-integrated energy distributions of proton emission induced by neutrons on  $^{93}\text{Nb}$  around the incident energy of 14 MeV have provided proton total-emission cross-sections at this energy. On the other hand, the overall account of the measured energy spectra, in the limit of the error bars (Figure 11D of Ref. [16]), have particularly supported an appropriate

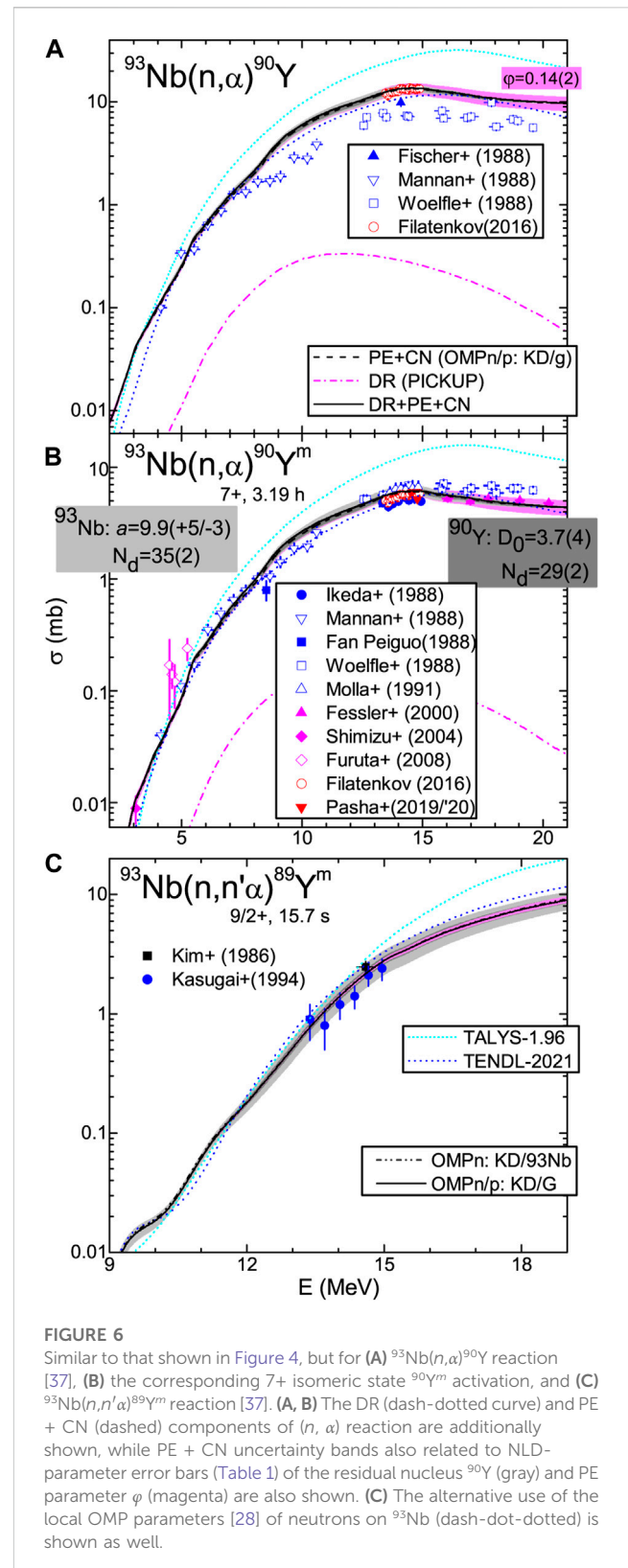


description of proton PE in neutron-induced reactions on  $^{93}\text{Nb}$  at least within this energy range. Moreover, there is a good agreement between the corresponding measured proton total-emission cross-sections and the model calculation results, as shown in Figure 5.

In this case, a comparison is possible between the NLD effects on calculated cross-sections due to NLD-parameter uncertainty related to either the adopted limits of the average  $a$ -value for the residual nucleus  $^{93}\text{Nb}$ , or the error bars of the fitted  $D_0^{\text{exp}}$  for the proton-emission residual  $^{93}\text{Zr}$ . While the numbers of low-lying levels also fitted for these, both odd  $A$  nuclei are rather similar, and the average  $a$ -value for  $^{93}\text{Nb}$  has the aforementioned rather large limits, whereas  $D_0^{\text{exp}}$  value for  $^{93}\text{Zr}$  has a 23% singular uncertainty (Table 1).

Under these conditions, the corresponding uncertainty bands of the calculated excitation functions (Figure 5) are both large compared to those for the  $(n, xn)$  reactions. Thus, the uncertainty band related to NLD parameters for  $^{93}\text{Nb}$  has a width going from 29%, at the incident energy of 14.8 MeV, to 19% at the maximum of the proton-emission excitation function, which is around 23 MeV. The width of the uncertainty band related to NLD parameters for  $^{93}\text{Zr}$ , from 28% to 25% between the same incident energies, is evidently larger at the higher energies. It thus pointed out the importance of the higher accuracy of the  $D_0^{\text{exp}}$  measurements for improvement of the calculated reaction cross-sections with consistent parameters sets.

On the other hand, changes due to the use of the neutron local OMP [28] are only between  $-2\%$  and  $4\%$  for the same incident energies from 14.8 to 23 MeV. Somewhat larger ones, from  $9\%$  to  $-6\%$ , correspond to the use of the local OMP [28] for protons. Nevertheless, while the former are yet within the error bars of the



measured proton total-emission cross-sections, the latter match similarly only to the larger earlier data errors.

The  $(n, \alpha)$  reaction analysis has already been discussed [16], with the main points only briefly mentioned hereafter. Thus, the

spectroscopic factors provided by an analysis of the  $\alpha$ -particle angular distributions from  $^{91}\text{Zr}(t,x\alpha)^{89,90}\text{Y}$  led to a significant DR component for only 14 levels up to  $\sim 2.5$  MeV excitation energy by the  $^{93}\text{Nb}(n,\alpha)^{90}\text{Y}$  reaction. Therefore, a minor pickup DR contribution has been found to the total cross-sections of this reaction in Figure 6A. The same is the case of the  $7^+$  isomer of the residual nucleus  $^{90}\text{Y}$ , which was also recently measured, as shown in Figure 6B, for energies from the effective threshold to above 20 MeV. On the other hand, the suitable account of the whole higher energy side of the emitted  $\alpha$ -spectrum around 14 MeV validated the value  $\varphi = 0.14$  of the aforementioned  $\alpha$ -particle PE parameter [3, 45]. Then, the agreement of the measured and CN + PE calculated cross-sections within the error bars of the recent data made possible the assessment of a PE uncertainty band corresponding to  $\Delta\varphi = 0.02$  [16]. The width of this uncertainty band increases from  $\sim 19\%$ , around the incident energy of 10 MeV, to  $\sim 30\%$  at 20 MeV.

Similar results were obtained for the  $7^+$  isomeric-state  $^{90}\text{Y}^m$  activation and that of the  $9/2^+$  isomer through the  $^{93}\text{Nb}(n,n'\alpha)^{89}\text{Y}^m$  reaction shown in Figure 6C. The only difference in the latter case is that there is no distinct PE uncertainty band. A related parallel with the accuracy of the measured data was thus not possible either, while the main aim to validate the  $\alpha$ -particle OMP [18] was entirely proved by the suitable CN + PE account of all  $\alpha$ -emission data.

Moreover, a comparison has been possible between the NLD effects due to uncertainties of the average  $a$ -value for the residual nucleus  $^{93}\text{Nb}$ , and the  $a$ -value of the residual  $^{90}\text{Y}$ , which corresponds to the error bar of the fitted  $D_0^{\text{exp}}$ . However, the numbers of low-lying levels also fitted for these nuclei are rather similar, and the average  $a$ -value for  $^{93}\text{Nb}$  has the aforementioned rather large limits whereas the  $D_0^{\text{exp}}$  value for  $^{90}\text{Y}$  has only 11% uncertainty (Table 1). Consequently, the uncertainty band related to NLD parameters for  $^{93}\text{Nb}$  has the width from around 26%–6%, for incident energies between 10 and 20 MeV. The width of the uncertainty band related to NLD parameters for  $^{90}\text{Y}$ , from  $\sim 8\%$ –6% between the same incident energies, is indeed smaller at lower energies. It is thus evident that there is a correlation between the accuracy of the HF model parameters, given in turn by the distinct data taken previously into account, and the final uncertainties of the calculated reaction cross-sections. So, the importance of  $D_0^{\text{exp}}$  measurements with a higher accuracy, for more accurate calculated reaction cross-sections using consistent parameter sets, is again pointed out.

Hence, larger calculated cross-section uncertainties are at lower energies, due to average level-density parameters, while rather similar values are related to the PE account at higher energies. Notably, the former aforementioned uncertainties become comparatively much lower as the energy increases, whereas the latter have significant values also at lower energies. Nevertheless, changes due to the use of local OMPs [28] for neutrons are higher than 1% only from the incident energy of 14 MeV and up to 2% at 20 MeV. At the same time, the local OMP [28] for protons led to the changes below 1% in the whole energy range.

Finally, the same analysis of the activation of  $9/2^+$  isomers through the  $^{93}\text{Nb}(n,n'\alpha)^{89}\text{Y}^m$  reaction makes possible a better understanding of the balance between the experimental and calculated cross-sectional uncertainties. Its enlarged illustration in Figure 6C facilitates the note of the calculated cross-section PE uncertainty band with a width from  $\sim 2\%$ –16% for incident energies between 10 and 20 MeV. However, it is well below the uncertainty band related to NLD parameters for  $^{93}\text{Nb}$ , with a width of around 23% at the same energy, but rather close to that related to NLD parameters for  $^{90}\text{Y}$ . For the sake of completeness, an increase due

to the use of the local OMP [28] for neutrons from 4%–2% within the same energy range should be mentioned. Therefore, one may see, in this case, a close matching of the experimental and calculated cross-section uncertainties and the latter truly corresponding to the limits of the distinct data that are previously involved within the consistent parameter assessment.

## 4 Conclusion and outlook

A concurrent assessment of all measured excitation functions for various reactions induced by neutrons on  $^{93}\text{Nb}$ , in addition to the results of using default parameters within the worldwide used computer code TALYS-1.96 [14] and the currently related TENDL-2021 evaluation [15], is reported. The use of consistent parameter sets that are formerly obtained or validated by analysis of other independent data is mainly concerned. Moreover, no empirical rescaling factors of the  $\gamma$  and/or neutron widths have been used.

A proper account of all available data for competitive reaction channels has prevented compensation effects of less accurate model parameters. On the other hand, detailed analyses based on consistent input parameter sets are really needed to eventually improve the global parameters for involvement in large-scale evaluations. Moreover, the consistency of input parameters may lead to a similar condition of the model calculation results, agreeing with the data obtained previously and always behaving in a similar way.

The correlation between the measured error bars of the primary data providing the consistent input parameters and the final uncertainty bands of the calculated results has been pointed out. Thus, e.g., comparison between the nuclear-level density effects due to the uncertainty of either the average-level density parameter  $a$  for the residual nucleus  $^{93}\text{Nb}$  or the fitted  $D_0^{\text{exp}}$  for the residual nuclei  $^{93}\text{Zr}$  and  $^{90}\text{Y}$  has been carried out.

Finally, matching of the experimental and calculated cross-section uncertainties has been obtained and its correspondence to the limits of the distinct data previously involved in the consistent parameter assessment. Remaining questions and the need for additional measurements are emphasized.

## Data availability statement

The original contributions presented in the study are included in the article. Further inquiries can be directed to the corresponding author.

## Author contributions

All authors listed have made a substantial, direct, and intellectual contribution to the work and approved it for publication.

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## Conflict of interest

The authors declare that the research was conducted in the absence of any commercial or financial relationships that could be construed as a potential conflict of interest.

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